1.0 Purpose

The following procedure applies to acceptance, handling, and release of end dump trailers at the White Mesa Mill (the "Mill"). International Uranium (USA) Corporation ("IUSA") receives material for processing, in either bulk or non-bulk packaging. This procedure addresses one form of bulk packaging – end dump trailers. This procedure may be amended, subject to approval by IUSA's Safety and Environmental Review Panel (SERP), from time to time as appropriate to address the individual requirements of specific feed materials, or projects.

2.0 Ore Receiving

1. Check truck scale for zero balance at the beginning of each shift.

2. When each truck driver enters the Restricted Area for the first time, the Scale house operator will provide hazard training for the driver. The driver will be provided with the Safety Training Form (copy attached). All drivers will be required to read the Safety Training Form and sign and date the Safety Training Form indicating that they understand and agree to follow IUSA's safety rules and procedures while on company property. The Scale house operator will sign the Safety Training Form as the instructor for IUSA. Completed Safety Training Forms will be turned in to the Safety Department for future reference.

3. Inspect all copies of the Bill of Lading (BOL) to ensure that the shipment is destined for the Mill and that all shipping documentation is in order (see Section 8.2). **If any discrepancies are noted notify the Mill management immediately. Do not empty the end dump trailer until all paperwork discrepancies are corrected.**

4. Assign next available shipment number and Mill load number to the inbound shipment. Record the Mill load number, inbound date and both the truck and end dump trailer numbers on the Scale house Weight Ticket (SWT).

5. Enter the loaded weight of the end dump truck and trailer on the SWT.

3.0 Ore Unloading

1. After weighing the truck and trailer, direct the driver to the specified ore storage pad area where the material will be unloaded. Prior to unloading the tarp is inspected for damage and then removed, if necessary. The driver is then directed to unload the material, ensuring all personnel are clear of the trailer and the immediate area.

2. After unloading the material replace the tarp, unless the trailer is being decontaminated for unrestricted release.

3. After the tarp has been replaced on the trailer, direct the driver back to the scales for an empty weight.

4. Record the empty weight on the appropriate SWT.

5. Use a front-end loader or similar equipment to push material into the designated ore lot pile.

4.0 Decontamination and Release of End Dump Trailers and Trucks

All end dump trailers and trucks will be decontaminated after unloading prior to leaving the Mill. Generators or transporters will notify IUSA whether a specific trailer is to be released for restricted or unrestricted use. Any trailers that are to be released for restricted use will be decontaminated according to the requirements contained in U.S. Department of Transportation (DOT) Part 49 CFR 173.441(b) and 173.443 (copies attached). Any trailers that are to be released for unrestricted use will be decontaminated according to the requirements found in Table 1 of the Nuclear Regulatory Commission's (NRC's) Policy and Guidance Directive FC-85-23, "Guidelines for Decontamination of Facilities and Equipment Prior to Release for Unrestricted Use or Termination of Licenses for Byproduct, Source, or Special Nuclear Material" issued May 1987 (copy attached). Trailers requiring repair will be decontaminated for unrestricted release, to facilitate repairs by the transporter at the transporter's own site. Trailers may be repaired without undergoing full decontamination if repaired within the Restricted Area of the Mill.

5.0 Decontamination and Release of Equipment for Restricted Use

1. After the tarp has been replaced on the trailer and the empty weight obtained, the driver will be directed to the decontamination pad.

2. Decontaminate the exterior of each trailer, truck and tires thoroughly, using a high-pressure water wash.

3. After the truck and trailer are decontaminated, the driver will be directed to the gate, along the decontamination route. The decontamination route is a graveled roadway specifically designed for decontaminated equipment to exit the Restricted Area. If it becomes necessary due to environmental conditions or residual mud in the tires etc., direct the driver to proceed along the decontamination route to the secondary wash station. Wash any visual residual mud off of the tires, or exterior surface of the truck and trailer. Otherwise, direct the driver to proceed along the decontamination route to the tertiary decontamination and final scanning area.

4. Contact a Radiation Technician to perform a radiological contamination survey for restricted release of the truck and trailer. If the Radiation Technician indicates areas on the truck or trailer that require further decontamination, decontaminate those areas as necessary.

5. The Radiation Technician or RSO will scan the trailer, truck, and tires in various locations as shown on the Equipment Survey for Restricted Release (copy attached) and document the scan readings on the Equipment Survey for Restricted Release. The contamination survey will be performed using appropriate radiological instrumentation for total activity in accordance with DOT transportation regulations. The release standards to be met for restricted release are contained in U.S. Department of Transportation (DOT) Part 49 CFR 173.441(b) and 173.443 (copies attached).

6. If the trailer, truck or tires do not meet the radiological release survey requirements or shows visually observable contamination, the truck and trailer will either be returned to the secondary decontamination pad for further decontamination or will be washed again at the tertiary decontamination area.

7. The Radiation Technician or RSO will fill out the Equipment Survey for Restricted Release form (copy attached) to document that the truck and trailer has been authorized for release for restricted use. These forms are filed in the Radiation Department. The Radiation Technician or RSO will place a white sticker on the trailer that says, "EMPTY" and "This package conforms to the conditions and limitations specified in 49 CFR 173.428 for radioactive material, excepted package – empty package, UN2910".

8. After a truck and trailer have been released, the driver will perform a visual inspection of the truck, trailer and tires.

9. Leaving his truck outside of the Restricted Area, the driver will return to the Scale House to pick up the documentation for the empty trailer.

6.0 Decontamination and Release of Equipment for Unrestricted Use

1. After the empty weight of the trailer has been obtained, the driver will be directed to the decontamination pad. The tarp will not be replaced on the trailer or, if necessary, the tarp shall be removed.

2. Open the trailer tailgate, elevate the trailer ensuring that there are no overhead hazards and decontaminate the trailer using a high-pressure water wash. Make sure to thoroughly wash the inside and outside of the trailer. The truck and tires shall also be washed.

3. After the truck and trailer are decontaminated, the driver is directed to follow the decontamination route. The decontamination route is a graveled roadway specifically designed for decontaminated equipment to exit the Restricted Area. If it becomes necessary due to environmental conditions or residual mud in the tires etc., the driver is instructed to proceed along the decontamination route to the secondary wash station. Wash any visual residual mud off of the tires, or exterior surface of the container. Otherwise, the driver is instructed to proceed along the decontamination route to the tertiary decontamination and final scanning area.

4. Contact a member of the Radiation Department staff to conduct the appropriate radiological survey for unrestricted release of the truck, trailer and tires. The release standards to be met for unrestricted release are contained in Table 1 of the Nuclear Regulatory Commission's (NRC's) Policy and Guidance Directive FC-85-23, "Guidelines for Decontamination of Facilities and Equipment Prior to Release for Unrestricted Use or Termination of Licenses for Byproduct, Source, or Special Nuclear Material" issued May 1987 (copy attached). The RSO will prepare a memorandum detailing the applicable standard on Table 1 for each project. The RSO will also provide the training of the Radiation Technicians required.

5. The Radiation Technician will perform a radiological contamination survey of the truck and trailer. If the Radiation Technician indicates areas that require further decontamination, decontaminate those areas as necessary.

6. If the truck, trailer and tires meet the radiological release survey and visual inspection requirements, the Radiation Technician will place a red sticker on the trailer that says, "THIS CONTAINER HAS BEEN FULLY DECONTAMINATED AND SURVEYED FOR "UNRESTRICTED USE" BY: (FILL IN NAME OF RADIATION TECHNICIAN)". The RSO or Radiation Technician that performed the release survey will then sign the red sticker and date it. In addition, the RSO or Radiation Technician will fill out a Decontamination Final Release Form (copy attached) to document that the truck and trailer have been cleared for unrestricted release. The Decontamination Release Form will be turned in to the Mill Administration Office daily for filing and distribution.

7. After a truck and trailer have been released, the driver will do a visual inspection of the truck and trailer.

8. Leaving his truck outside of the Restricted Area, the driver will return to the Scale House to pick up the documentation for the empty trailer.

7.0 Hazard Identification and Safety

7.1 Required Personal Protective Equipment (PPE)

In all areas of the Mill covered by this procedure, hard hats, safety glasses and steel-toed shoes are required at a minimum. These must be worn in all areas of the Mill with the exception of the Administration Building.

7.2 Industrial Hazards and Safety

1. Use caution when the trailers are backing onto the Ore Pad.

2. Ensure that all personnel within 50 feet of the area where the end dump trailer are aware that unloading is about to commence. Move at least 25 feet away from the rear of the trailer during the initial unloading operation.

3. Drivers must use caution during the unloading process and be aware of any overhead hazards.

4. Do not place any part of your body inside the trailer when the trailer is being tipped and the tailgate is open. Only work around the tailgate after it has been properly blocked open.

5. Be aware of high-pressure wash water.

6. Be aware of slippery conditions on the ore pad during periods of inclement weather.

7. Be aware of the potential for ice build-up on and around the decontamination pad during periods of cold weather.

8. Use caution when entering or exiting equipment. Be sure to use the ladders and hand rails. **Do not jump off of the equipment.**

9. Always use a ladder when entering and/or exiting the interior of an end dump trailer.

8.0 Paperwork Tracking

1. Each trailer will have a unique sequential project number assigned to it at the generating facility. This number will be entered onto the Bill of Lading (BOL) and attached to the trailer prior to shipment from the generation site.

2. Upon arrival at the Mill, the truck driver will turn in all of his/her paperwork to the Scale House operator who will verify that the BOL number, trailer number and project number assigned to the shipment match on all copies of the BOL. The Scale House operator will also verify that the actual trailer number matches the BOL Trailer Number. **If there are any discrepancies in any of the numbers notify Mill management immediately.** Only original paperwork will be accepted. If the original paperwork does not come with the trailer, **notify Mill management immediately**. The Scale House operator will sign the BOL, acknowledging receipt of the material at the Mill, **if all of the paperwork is in order**. Depending on contractual and/or sampling requirements, final acceptance or rejection of certain alternate feed materials may be contingent on analytical results.

3. Each trailer will be transported across the scale at the Mill prior to and after being unloaded. The appropriate information will be entered into the project database. All copies of the SWTs and BOLs will be forwarded to the Mill Records Manager on a daily basis or other frequency specified by Mill Management, from time to time.

4. The Mill Records Manager will compile and reconcile the BOL's and SWTs for distribution.

SAFETY TRAINING FOR ALTERNATE FEED DELIVERY PERSONNEL

Welcome to International Uranium (USA) Corporation's, White Mesa Mill. In order to assure your safety while on our property, we would like to acquaint you with the safety rules and procedures, which you will be required to follow while on our property.

1.0 General Safety

1. Approved hard hats and safety glasses are required at all times except when inside the cab of your truck.

2. This is a smoke free facility. No smoking is allowed on the property. Eating anything, drinking, chewing candy, gum or tobacco is also not allowed in .the Mill restricted area due to radiation hazards.

3. All mobile equipment will not exceed a speed limit of 5 miles per hour.

4. Be aware of the possibility of a truck turning over while dumping. Ensure that the truck is on level ground and brakes are set prior to dumping.

5. Check for potential overhead hazards prior to dumping.

6. If material is hung up in the trailer bed, it is not permissible to work in the bed while it is in the dump position. If it is necessary to get in the bed of the trailer to free a hang up, the bed must be lowered.

7. Be aware of slippery conditions on the ore pad during periods of inclement weather.

8. Be aware of the potential for ice build-up on and around the decontamination pad during periods of cold weather.

9. Use caution when entering or exiting equipment.

2.0 Radiation Safety

1. All drivers are required to scan for alpha radiation prior to leaving the Mill restricted area.

2. All equipment, i.e. trucks and trailers, will be scanned for radiation prior to leaving the Mill's restricted area.

_____ _____

Driver (Printed) Scale House Operator

_____ _____

Driver (Signature) Date

[Code of Federal Regulations]
[Title 49, Volume 2, Parts 100 to 185]
[Revised as of October 1, 1999]
From the U.S. Government Printing Office via GPO Access
[CITE: 49CFR173.441]

TITLE 49--TRANSPORTATION

CHAPTER I--RESEARCH AND SPECIAL PROGRAMS ADMINISTRATION, DEPARTMENT OF TRANSPORTATION

PART 173--SHIPPERS--GENERAL REQUIREMENTS FOR SHIPMENTS AND PACKAGINGS--Table of Cont

Subpart I--Class 7 (Radioactive) Materials

Sec. 173.441 Radiation level limitations.

(a) Except as provided in paragraph (b) of this section, each package of Class 7 (radioactive) materials offered for transportation must be designed and prepared for shipment, so that under conditions normally incident to transportation, the radiation level does not exceed 2 mSv/hour (200 mrem/hour) at any point on the external surface of the package, and the transport index does not exceed 10.

(b) A package which exceeds the radiation level limits specified in paragraph (a) of this section must be transported by exclusive use shipment, and the radiation levels for such shipment may not exceed the following during transportation:

(1) 2 mSv/h (200 mrem/h) on the external surface of the package unless the following conditions are met, in which case the limit is 10 mSv/h (1000 mrem/h):

(i) The shipment is made in a closed transport vehicle;

(ii) The package is secured within the vehicle so that its position remains fixed during transportation; and

(iii) There are no loading or unloading operations between the beginning and end of the transportation;

(2) 2 mSv/h (200 mrem/h) at any point on the outer surfaces of the vehicle, including the top and underside of the vehicle; or in the case of a flat-bed style vehicle, at any point on the vertical planes projected from the outer edges of the vehicle, on the upper surface of the load or enclosure if used, and on the lower external surface of the vehicle;

(3) 0.1 mSv/h (10 mrem/h) at any point 2 meters (6.6 feet) from the outer lateral surfaces of the vehicle (excluding the top and underside of the vehicle); or in the case of a flat-bed style vehicle, at any point 2 meters (6.6 feet) from the vertical planes projected by the outer edges of the vehicle (excluding the top and underside of the vehicle); and

(4) 0.02 mSv/h (2mrem/h) in any normally occupied space, except that this provision does not apply to carriers if they operate under the provisions of a State or federally regulated radiation protection program and if personnel under their control who are in such an occupied space wear radiation dosimetry devices.

(c) For shipments made under the provisions of paragraph (b) of this section, the offeror shall provide specific written instructions for maintenance of the exclusive use shipment controls to the carrier. The instructions must be included with the shipping paper information. The instructions must be sufficient so that, when followed, they will cause the carrier to avoid actions that will unnecessarily delay delivery or unnecessarily result in increased radiation levels or radiation exposures to transport workers or members of the general public.

(d) Packages exceeding the radiation level or transport index prescribed in paragraph (a) of this section may not be transported by aircraft.

[Amdt. 173-244, 60 FR 50307, Sept. 28, 1995, as amended at 63 FR 48568, Sept. 10, 1998]

[Code of Federal Regulations]
[Title 49, Volume 2, Parts 100 to 185]
[Revised as of October 1, 1999]
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[CITE: 49CFR173.443]

[Page 583-584]

TITLE 49--TRANSPORTATION

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PART 173--SHIPPERS--GENERAL REQUIREMENTS FOR SHIPMENTS AND PACKAGINGS--Table of Cont

Subpart I--Class 7 (Radioactive) Materials

Sec. 173.443 Contamination control.

(a) The level of non-fixed (removable) radioactive contamination on the external surfaces of each package offered for transport must be kept as low as reasonably achievable. The level of non-fixed radioactive contamination may not exceed the limits set forth in table 11 and must be determined by either:

(1) Wiping an area of 300 square centimeters of the surface concerned with an absorbent material, using moderate pressure, and measuring the activity on the wiping material. Sufficient measurements must be taken in the most appropriate locations to yield a representative assessment of the non-fixed contamination levels. The amount of radioactivity measured on any single wiping material, when averaged over the surface wiped, may not exceed the limits set forth in table 11 at any time during transport; or

(2) Using other methods of assessment of equal or greater efficiency, in which case the efficiency of the method used must be taken into account and

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the non-fixed contamination on the external surfaces of the package may not exceed ten times the limits set forth in table 11, as follows:

Table 11--Non-Fixed External Radioactive Contamination-Wipe Limits

Contaminant	Maximum permissible limits	
	Bq/cm²	uCi/cm²
Beta and gamma emitters and low toxicity alpha emitters	0.4	10⁻⁵ 22
All other alpha emitting radionuclides	0.04	10⁻⁶ 2.2

(b) Except as provided in paragraph (d) of this section, in the case of packages transported as exclusive use shipments by rail or public highway only, the removable (non-fixed) radioactive contamination on any package at any time during transport may not exceed ten times the levels prescribed in paragraph (a) of this section. The levels at the beginning of transport may not exceed the levels prescribed in paragraph (a) of this section.

(c) Except as provided in paragraph (d) of this section, each transport vehicle used for transporting Class 7 (radioactive) materials as an exclusive use shipment that utilizes the provisions of paragraph (b) of this section must be surveyed with appropriate radiation detection instruments after each use. A vehicle may not be returned to service until the radiation dose rate at each accessible surface is

0.005 mSv per hour (0.5 mrem per hour) or less, and there is no significant removable (non-fixed) radioactive surface contamination as specified in paragraph (a) of this section.

(d) Paragraphs (b) and (c) of this section do not apply to any closed transport vehicle used solely for the transportation by highway or rail of Class 7 (radioactive) material packages with contamination levels that do not exceed 10 times the levels prescribed in paragraph (a) of this section if--

(1) A survey of the interior surfaces of the empty vehicle shows that the radiation dose rate at any point does not exceed 0.1 mSv per hour (10 mrem per hour) at the surface or 0.02 mSv per hour (2 mrem per hour) at 1 meter (3.3 feet) from the surface;

(2) Each vehicle is stenciled with the words ``For Radioactive Materials Use Only'' in letters at least 76 millimeters (3 inches) high in a conspicuous place on both sides of the exterior of the vehicle; and

(3) Each vehicle is kept closed except for loading or unloading.

[Amdt. 173-244, 60 FR 50307, Sept. 28, 1995, as amended by Amdt. 173-244, 61 FR 20753, May 8, 1996]

GUIDELINES FOR DECONTAMINATION OF FACILITIES AND EQUIPMENT PRIOR TO RELEASE FOR UNRESTRICTED USE OR TERMINATION OF LICENSES FOR BYPRODUCT, SOURCE, OR SPECIAL NUCLEAR MATERIAL

U.S. Nuclear Regulatory Commission
Division of Fuel Cycle, Medical, Academic,
and Commercial Use Safety
Washington, DC 20555

April 1993

The instructions in this guide, in conjunction with Table 1, specify the radionuclides and radiation exposure rate limits which should be used in decontamination and survey of surfaces or premises and equipment prior to abandonment or release for unrestricted use. The limits in Table 1 do not apply to premises, equipment, or scrap containing induced radioactivity for which the radiological considerations pertinent to their use may be different. The release of such facilities or items from regulatory control is considered on a case-by-case basis.

1. The licensee shall make a reasonable effort to eliminate residual contamination.

2. Radioactivity on equipment or surfaces shall not be covered by paint, plating, or other covering material unless contamination levels, as determined by a survey and documented, are below the limits specified in Table 1 prior to the application of the covering. A reasonable effort must be made to minimize the contamination prior to use of any covering.

3. The radioactivity on the interior surfaces of pipes, drain lines, or ductwork shall be determined by making measurements at all traps, and other appropriate access points, provided that contamination at these locations is likely to be representative of contamination on the interior of the pipes, drain lines, or ductwork. Surfaces of premises, equipment, or scrap which are likely to be contaminated but are of such size, construction, or location as to make the surface inaccessible for purposes of measurement shall be presumed to be contaminated in excess of the limits.

4. Upon request, the Commission may authorize a licensee to relinquish possession or control of premises, equipment, or scrap having surfaces contaminated with materials in excess of the limits specified. This may include, but would not be limited to, special circumstances such as razing of buildings, transfer of premises to another organization continuing work with radioactive materials, or conversion of facilities to a long-term storage or standby status. Such requests must:

   a. Provide detailed, specific information describing the premises, equipment or scrap, radioactive contaminants, and the nature, extent and degree of residual surface contamination.

   b. Provide a detailed health and safety analysis which reflects that the residual amounts of materials on surface areas, together with other considerations such as prospective use of the premises, equipment, or scrap, are unlikely to result in an unreasonable risk to the health and safety of the public.

5. Prior to release of premises for unrestricted use, the licensee shall make a comprehensive radiation survey which establishes that contamination is within the limits specified in Table 1. A copy of the survey report shall be filed with the Division of Fuel Cycle Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and also the Administrator of the NRC Regional Office having jurisdiction. The reports should be filed at least 30 days prior to the planned date of abandonment. The survey report shall:

   a. Identify the premises.

   b. Show that reasonable effort has been made to eliminate residual contamination.

   c. Describe the scope of the survey and general procedures followed.

   d. State the findings of the survey in units specified in the instructions.

   Following review of the report, the NRC will consider visiting the facilities to confirm the survey.

TABLE 1

ACCEPTABLE SURFACE CONTAMINATION LEVELS

NUCLIDES[a]	AVERAGE[b,c,f]	MAXIMUM[b,d,f]	REMOVABLE[b,e,f]
U-nat, U-235, U-238, and associated decay products	5,000 dpm α/100 cm^2	15,000 dpm α/100 cm^2	1,000 dpm α/100 cm^2
Transuranics, Ra-226, Ra-228, Th-230, Th-228, Pa-231, Ac-227, I-125, I-129	100 dpm/100 cm^2	300 dpm/100 cm^2	20 dpm/100 cm^2
Th-nat, Th-232, Sr-90, Ra-223, Ra-224, U-232, I-126, I-131, I-133	1,000 dpm/100 cm^2	3,000 dpm/100 cm^2	200 dpm/100 cm^2
Beta-gamma emitters (nuclides with decay modes other than alpha emission or spontaneous fission) except Sr-90 and others noted above.	5,000 dpm $\beta\gamma$/100 cm^2	15,000 dpm $\beta\gamma$/100 cm^2	1,000 dpm $\beta\gamma$/100 cm^2

[a] Where surface contamination by both alpha- and beta-gamma-emitting nuclides exists, the limits established for alpha- and beta-gamma-emitting nuclides should apply independently.

[b] As used in this table, dpm (disintegrations per minute) means the rate of emission by radioactive material as determined by correcting the counts per minute observed by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.

[c] Measurements of average contaminant should not be averaged over more than 1 square meter. For objects of less surface area, the average should be derived for each such object.

[d] The maximum contamination level applies to an area of not more than 100 cm^2.

[e] The amount of removable radioactive material per 100 cm^2 of surface area should be determined by wiping that area with dry filter or soft absorbent paper, applying moderate pressure, and assessing the amount of radioactive material on the wipe with an appropriate instrument of known efficiency. When removable contamination on objects of less surface area is determined, the pertinent levels should be reduced proportionally and the entire surface should be wiped.

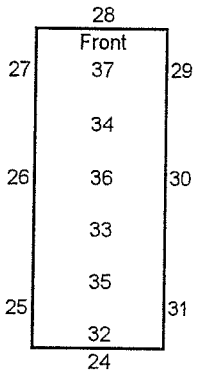
[f] The average and maximum radiation levels associated with surface contamination resulting from beta-gamma emitters should not exceed 0.2 mrad/hr at 1 cm and 1.0 mrad/hr at 1 cm, respectively, measured through not more than 7 milligrams per square centimeter of total absorber.

International Uranium (USA) Corporation

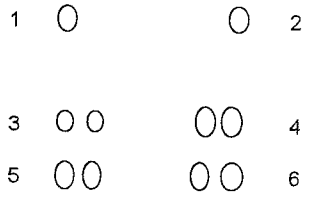
Equipment Survey for Restricted Release

Date: _____ **Container ID:** _____ **Surveyed By:** _____

Outside Locations



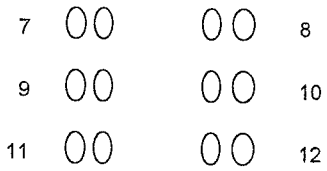
Truck and Tire Survey



Location #	Total Alpha/ Beta-Gamma cpm
24	
25	
26	
27	
28	
29	
30	
31	
32	
33	
34	
35	
36	
37	
Rails	

Location #	Total Alpha/ Beta-Gamma cpm
1:	
2:	
3:	
4:	
5:	
6:	
Truck Ext.:	

TRAILER TIRES



Instrument Data

Total Alpha/ Beta/Gamma

Model #: _____
SN: _____
Cal. Date: _____
Source: _____
Efficiency: _____

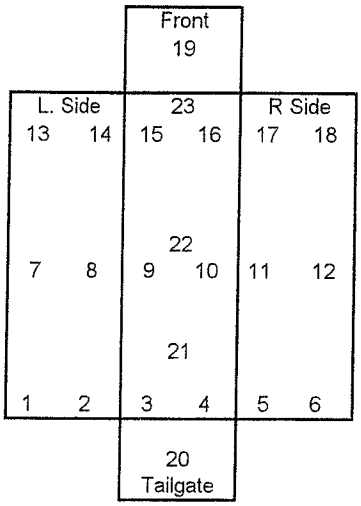
Location #	Total Alpha/ Beta-Gamma cpm
7	
8	
9	
10	
11	
12	
Chassis Ext.	

International Uranium (USA) Corporation

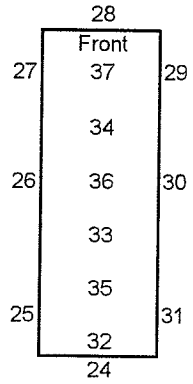
Equipment Survey for Unrestricted Release

Date: _____ **Container ID:** _____ **Surveyed By:** _____

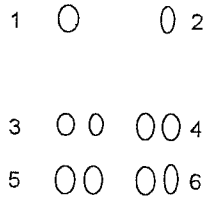
Inside Locations



Outside Locations



Truck and Tire Survey



dpm/100cm²

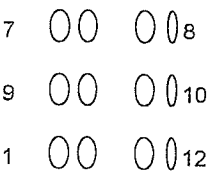
Location #	Total Alpha dpm/100cm²	Removable Alpha dpm/100cm²
1		
2		
3		
4		
5		
6		
7		
8		
9		
10		
11		
12		
13		
14		
15		
16		
17		
18		
19		

Location #	Total Alpha dpm/100cm²	Removable Alpha dpm/100cm²
20		
21		
22		
23		
24		
25		
26		
27		
28		
29		
30		
31		
32		
33		
34		
35		
36		
37		
Rails		

Total Alpha/ Beta-Gamma

Location	cpm
1:	
2:	
3:	
4:	
5:	
6:	
Truck Ext.:	

TRAILER TIRES



Total Alpha/ Beta-Gamma

Location	cpm
7	
8	
9	
10	
11	
12	
Chassis Ext.	

any location exceeds 1000 dpm/100 cm2 total alpha, a swear/wipe survey for removable alpha must be performed.

vever, additional removable aplha swipes, although not required, may be performed.

Instrument Data

Total Alpha

Instrument: ESP-1 / AC-3
SN 02299 / 1
02286 / 2
Cal. Date: _____
Function Check (5 x 1 min.)
Th 230 @ 30300 dpm
Bkg Average: _____
Dpm Average: _____

Removable Alpha

Model: _____
SN: _____
Cal. Date: _____
Function Check (5 x 1 min.)
Alpha Bkg Ave: _____
Alpha eff: _____
Alpha Factor: _____

Total Beta/Gamma

Model #: _____
SN: _____
Cal. Date: _____
Source: _____
Efficiency: _____

INTERNATIONAL URANIUM (USA) CORPORATION

DECONTAMINATION FINAL RELEASE

I have verified that tractor NA and/or container
(Tractor Number)
_____ has been checked for any contamination and has been
(Container Number)
authorized for final release.

_____	Radiation Technician	_____
Radiation Department	Title	Date